

LA-4574

c. 3

ENDF-141

EANDC(US)-151-A

**LOS ALAMOS SCIENTIFIC LABORATORY**  
of the  
**University of California**  
LOS ALAMOS • NEW MEXICO

Evaluated Nuclear Data for  
Hydrogen in the ENDF/B-II Format



This report was prepared as an account of work sponsored by the United States Government. Neither the United States nor the United States Atomic Energy Commission, nor any of their employees, nor any of their contractors, subcontractors, or their employees, makes any warranty, express or implied, or assumes any legal liability or responsibility for the accuracy, completeness or usefulness of any information, apparatus, product or process disclosed, or represents that its use would not infringe privately owned rights.

This report expresses the opinions of the author or authors and does not necessarily reflect the opinions or views of the Los Alamos Scientific Laboratory.

Printed in the United States of America. Available from  
National Technical Information Service  
U. S. Department of Commerce  
Springfield, Virginia 22151

Price: Printed Copy \$3.00; Microfiche \$0.65

Written: December 1970

Distributed: February 1971

LA-4574  
ENDF-141  
EANDC(US)-151-A  
UC-34, PHYSICS  
TID-4500

**LOS ALAMOS SCIENTIFIC LABORATORY**  
of the  
**University of California**  
LOS ALAMOS • NEW MEXICO

Evaluated Nuclear Data for  
Hydrogen in the ENDF/B-II Format



by

L. Stewart  
R. J. LaBauve  
P. G. Young



## EVALUATED NUCLEAR DATA FOR HYDROGEN IN THE ENDF/B-II FORMAT

by

L. Stewart, R. J. LaBauve, and P. G. Young

### ABSTRACT

The following nuclear data are given for hydrogen in the energy range from  $1.0 \times 10^{-5}$  eV to 20.0 MeV.

- File 1. The general information file includes a brief description of the data to follow.
- File 2. Values for nuclear spin and effective scattering radius are given in the resonance file.
- File 3. Smooth cross-section data are given for the total cross section, the free-atom elastic scattering cross section, and the radiative capture cross section; data for  $\bar{\mu}$ ,  $\xi$ , and  $\gamma$  are also included.
- File 4. The angular distributions for elastic scattering are given as probability vs cosine of the scattering angle.
- File 7. The free-atom-scattering cross section is the only information provided at thermal.
- File 12. Secondary gamma-ray production multiplicities for capture, which are equal to one, are given in this file.
- File 14. Gamma-ray angular distributions are provided for the single radiative capture gamma ray.

### INTRODUCTION

This evaluation for hydrogen (MAT = 1148) differs from the previous ENDF/B evaluation (MAT = 1001) in that the elastic scattering data were taken from recent work by Hopkins and Breit<sup>1</sup> and the data for radiative capture were taken from recent work by Horsley.<sup>2</sup> Also, gamma-ray production data, not given in the MAT = 1001 evaluation, are included. A complete listing for MAT = 1148 is given in the Appendix.

#### FILE 1: GENERAL INFORMATION

A brief summary of the data to follow is given in File 1. The atomic mass for hydrogen was taken to be 1.007825 from the May 1969 "Chart of the Nuclides."<sup>3</sup>

#### FILE 2: RESONANCE INFORMATION

Nuclear spin and effective scattering radius are given in this file. An effective scattering

radius of  $1.2756 \times 10^{-12}$  cm is consistent with a potential scattering cross section of 20.449 b, as determined from  $4\pi a^2$ . Singlet and triplet scattering radii are not included.

#### FILE 3: SMOOTH CROSS SECTIONS

Total cross sections (MT = 1) were obtained by adding the elastic scattering and radiative capture cross sections at all energies ( $1.0 \times 10^{-5}$  eV to 20.0 MeV). The hydrogen total cross sections are shown in Fig. 1.

The elastic scattering cross sections (MT = 2) were taken from an extensive theoretical treatment of fast neutron measurements by Hopkins and Breit.<sup>1</sup> In this work, a consistent set of cross sections and angular distributions were obtained by using a set of phase shifts previously determined at Yale University.<sup>4</sup> Tabular values of the elastic scattering cross section are given in Ref. 1 for only a

few energies, the two lowest points being 100 and 200 keV. The phase shift program and the Yale phase shifts were provided by Hopkins<sup>1</sup> so that many intermediate points could be calculated. At 0.1 keV, the lowest energy recommended for running this program, the scattering cross section is 20.4488 b. This value is in excellent agreement with the thermal cross section ( $20.442 \pm 0.023$  b) derived by Davis and Barschall<sup>5</sup> from a revised value of the effective range obtained by determining the best values of the neutron energies from many experiments below 5 MeV performed since 1950. Therefore, for this evaluation, the free-atom-scattering cross section is assumed to be constant below 100 eV and equal to the value calculated from the Yale phase shifts at 100 eV, giving a thermal cross section of 20.449 b. At higher energies, these theoretical predictions are in excellent agreement with the recent measurements of Davis<sup>6</sup> giving an average value of 0.84 for the square of the deviation for energies below 20.0 MeV. The elastic cross section for hydrogen from  $1.0 \times 10^{-5}$  eV to 20.0 MeV is shown in Fig. 2.

The cross sections for radiative capture (MT = 102) were taken from the 1966 publication of Horsley,<sup>2</sup> where a value of 332 mb was adopted for the thermal value. Deuteron photodisintegration cross sections were also employed in deriving radiative capture in Horsley's report. Although the Nuclear Data article by Horsley<sup>2</sup> was referenced for MAT = 1001, the values were taken from an early version described in AWRE O-23/65, and these were later revised for the Nuclear Data article. The latter report (Ref. 2) has been used for this evaluation, as suggested by Horsley. The radiative capture cross section for MAT = 1148 from  $1.0 \times 10^{-5}$  eV to 20.0 MeV is shown in Fig. 3.

The average value of the cosine in the laboratory system ( $\bar{\mu}_L$ ) for elastic scattering (MT = 251) was derived from the secondary angular distributions in File 4 (MT = 4). Values for  $\bar{\mu}_L$  from  $1.0 \times 10^{-5}$  eV to 20.0 MeV are shown in Fig. 4.

Values for  $\xi$ , the average logarithmic energy change per collision (MT = 252), and for  $\gamma$ , the Goertzel-Greuling constant (MT = 253), are taken equal to 1 over the range  $1.0 \times 10^{-5}$  eV to 20.0 MeV, following the MT = 1001 evaluation.

#### FILE 4: SECONDARY ANGULAR DISTRIBUTIONS

Angular distributions of secondary neutrons resulting from elastic scattering are tabulated from  $1.0 \times 10^{-5}$  eV to 20.0 MeV. Distributions at 0.1, 5, 10, 20, and 30 MeV are provided by Ref. 1; additional and intermediate data were calculated by using the Hopkins-Breit phase shift program and the Yale phase shifts. As shown in Figs. 5 through 16, the angular distributions above 100 keV are neither isotropic below 10 MeV, nor are they symmetric about  $90^\circ$  at higher energies as assumed in the earlier version (MAT = 1001). At 100 keV, the angular distributions are assumed to be isotropic because the  $180/0^\circ$  ratio is very nearly unity (1.0011). At 500 keV, this ratio approaches 1.005; therefore, the pointwise normalized probabilities as a function of the cosine of the scattering angle are provided at  $1.0 \times 10^{-5}$  eV (isotropic), 100 keV (isotropic), 500 keV, and at 1-MeV intervals from 1 to 20 MeV.

#### FILE 5: THERMAL DATA

Free-atom cross sections specified from  $1.0 \times 10^{-5}$  eV to 5 eV are included in this file.

#### FILE 12: PHOTON PRODUCTION CROSS SECTIONS

A multiplicity representation is used to describe the single hydrogen radiative capture gamma ray from  $1.0 \times 10^{-5}$  eV to 20.0 MeV. The multiplicity is referred to MT = 102 in File 3 and is unity at all neutron energies. To adequately represent the gamma-ray energy for MeV-incident neutrons, the neutron energy region from 0.2 to 20 MeV is divided into 16 different energy bands, and the gamma-ray energy is tabulated for each neutron energy band as

$$\bar{E}_\gamma = 2.225 \times 10^6 + \bar{E}_n/2 \quad (\text{eV}),$$

where  $\bar{E}_n$  is the neutron energy at the midpoint of the band in eV. The value  $2.225 \times 10^6$  eV corresponds to the deuteron binding energy; that is, the small energy change due to the nuclear recoil that accompanies gamma emission has been ignored.

#### FILE 14: GAMMA-RAY ANGULAR DISTRIBUTIONS

The gamma-ray angular distributions are assumed to be isotropic at all neutron energies from  $1.0 \times 10^{-5}$  eV to 20.0 MeV.

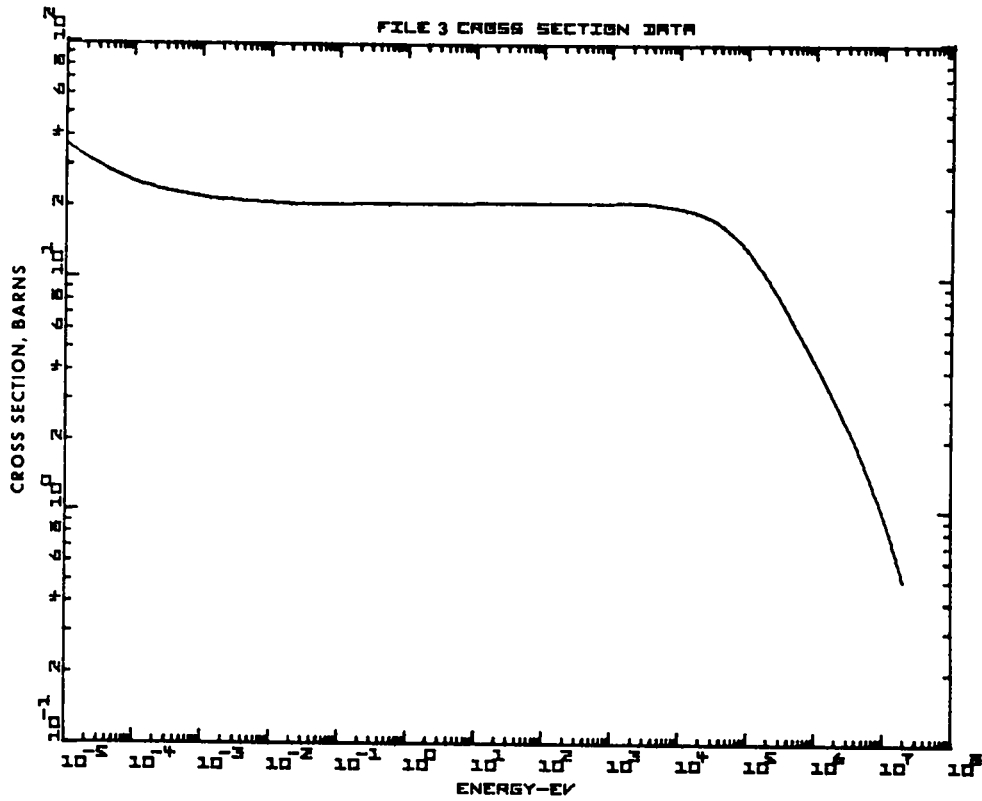


Fig. 1. Total cross section (MT = 1).

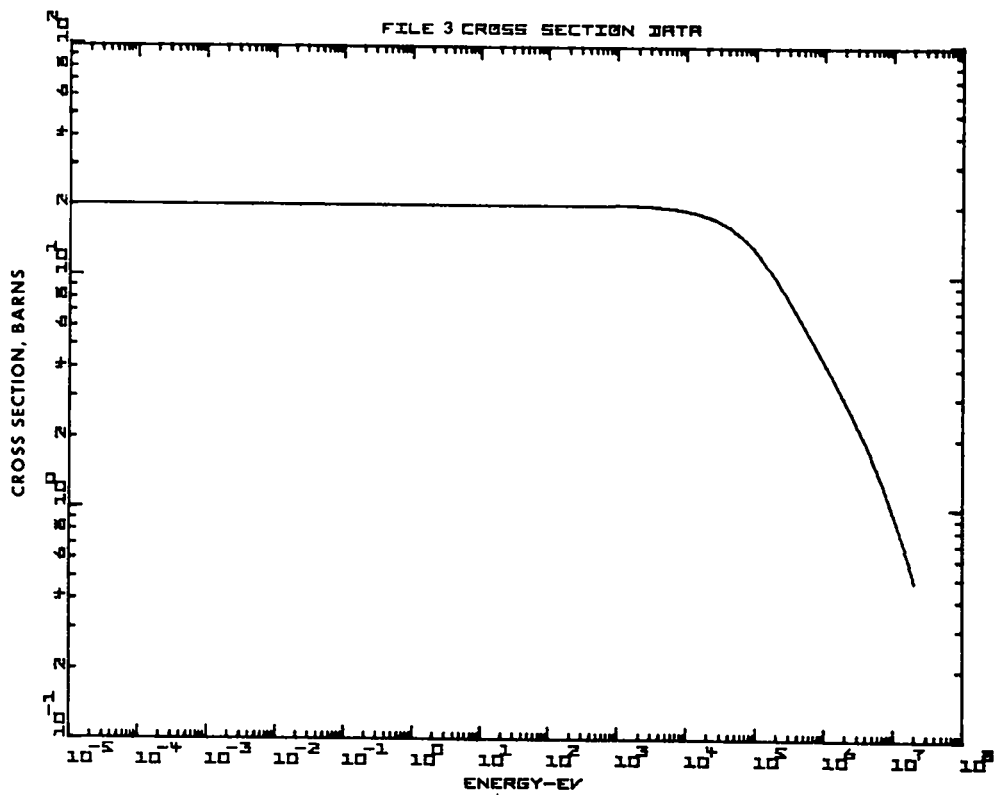


Fig. 2. Elastic scattering cross section (MT = 2).

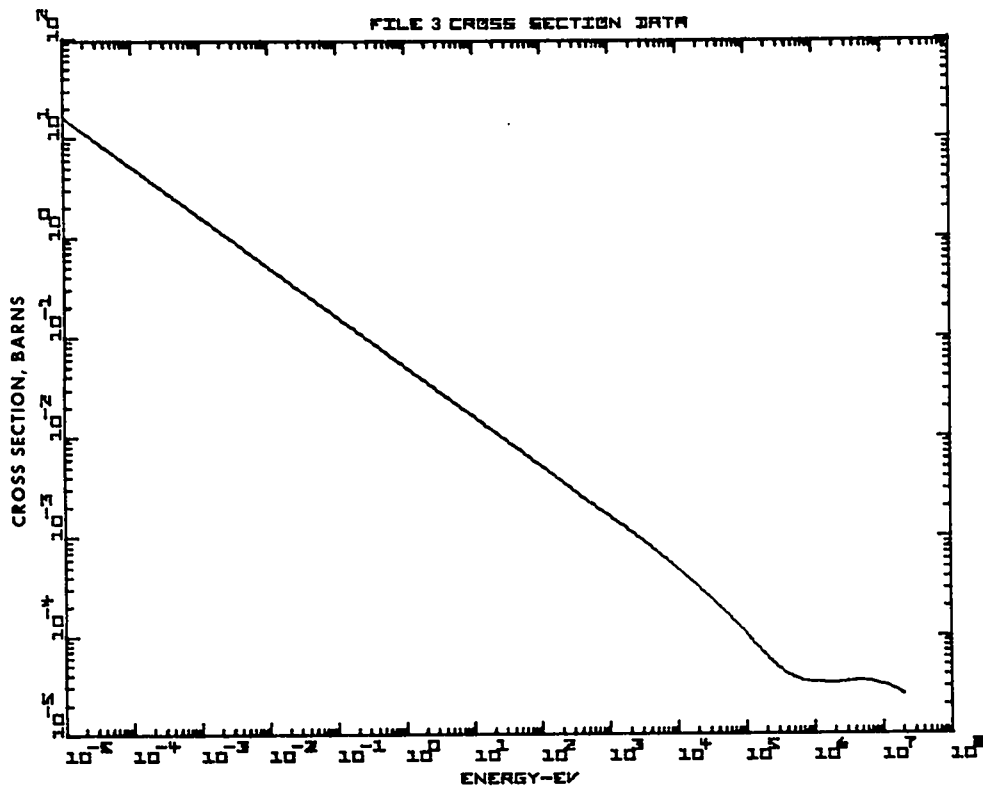


Fig. 3. Radiative capture cross section (MT = 102).

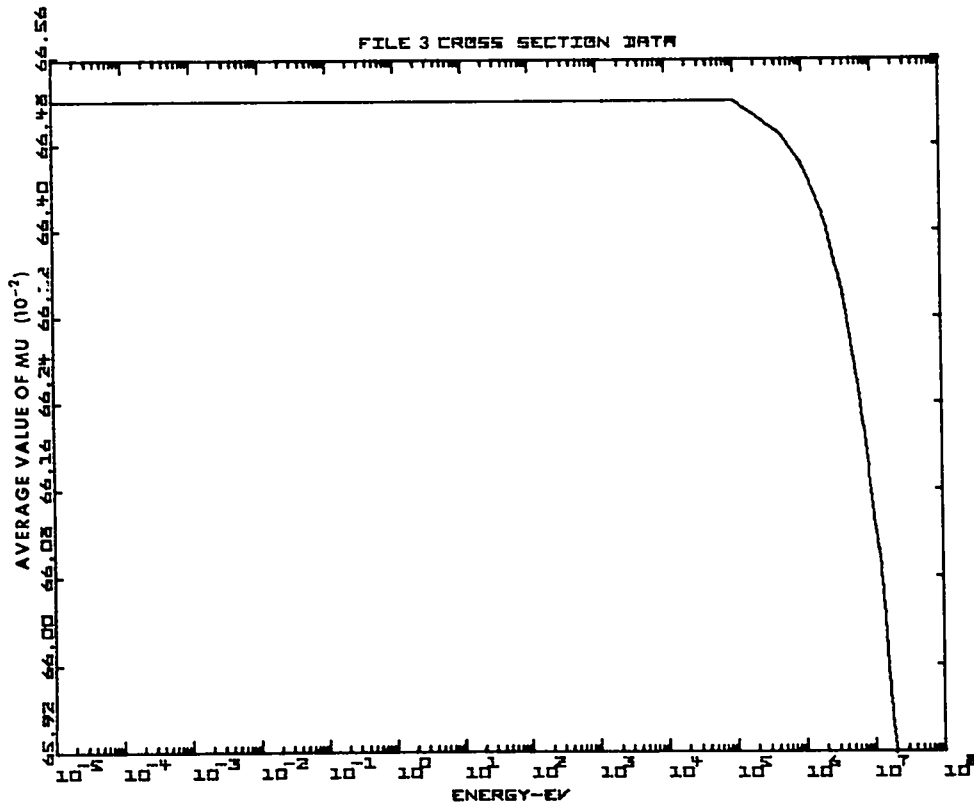


Fig. 4. Average value of cosine in laboratory system (MT = 251).

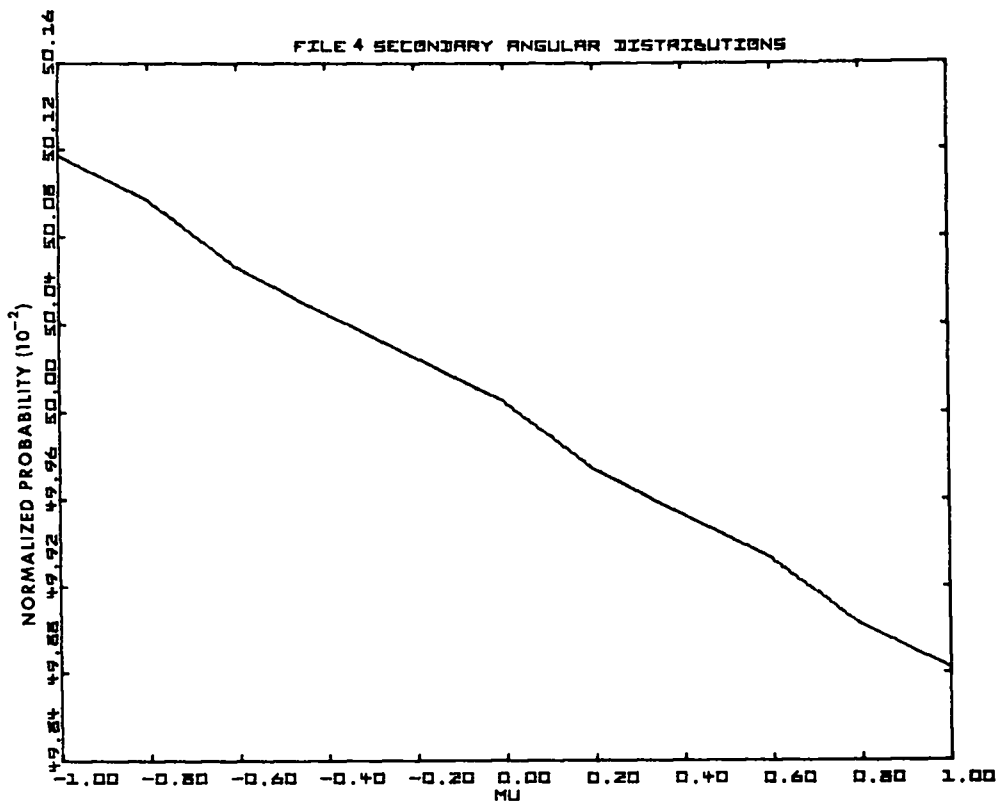


Fig. 5. Angular distribution for 0.5 MeV.

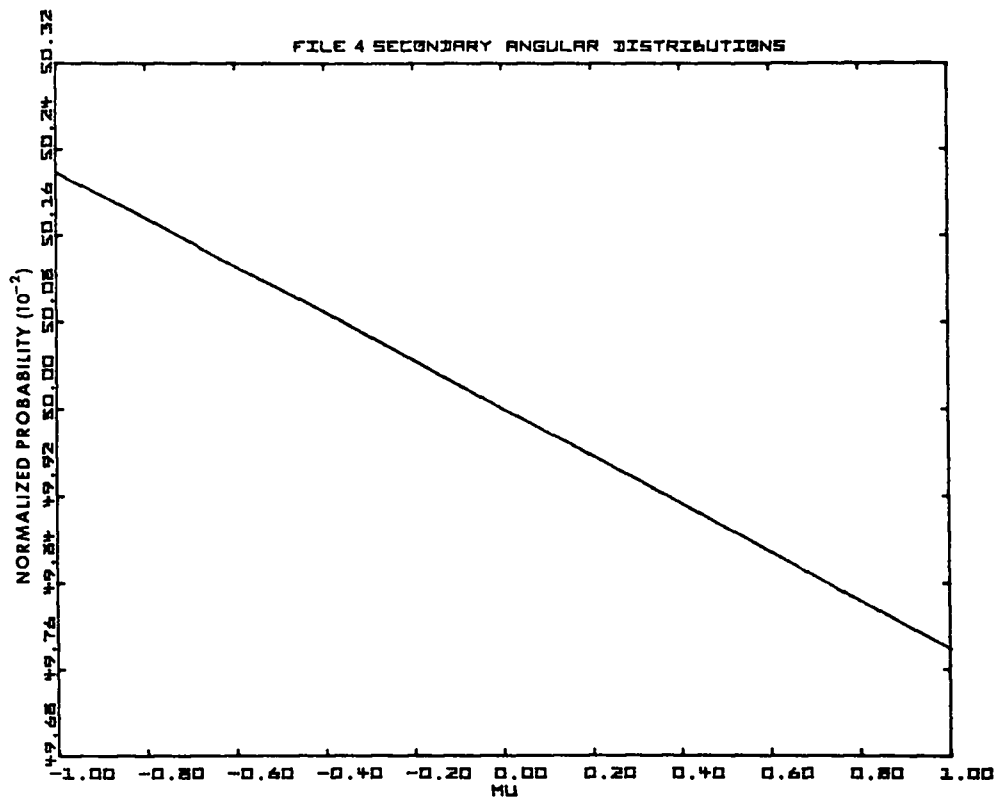


Fig. 6. Angular distribution for 1.0 MeV.



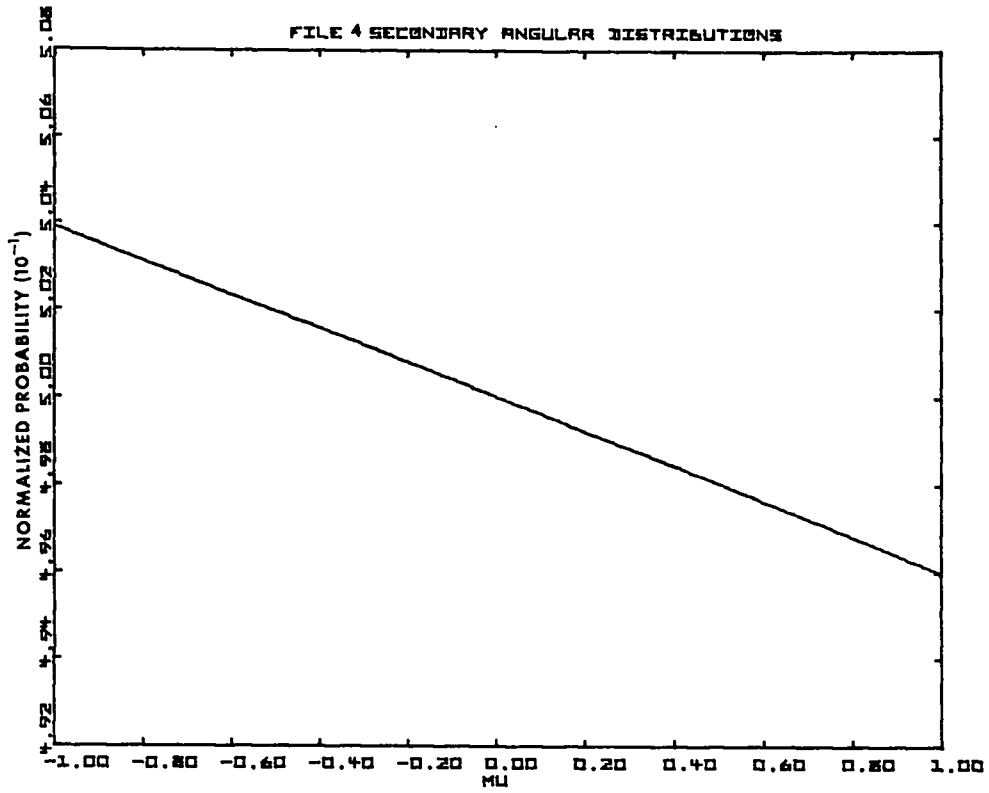


Fig. 7. Angular distribution for 2.0 MeV.

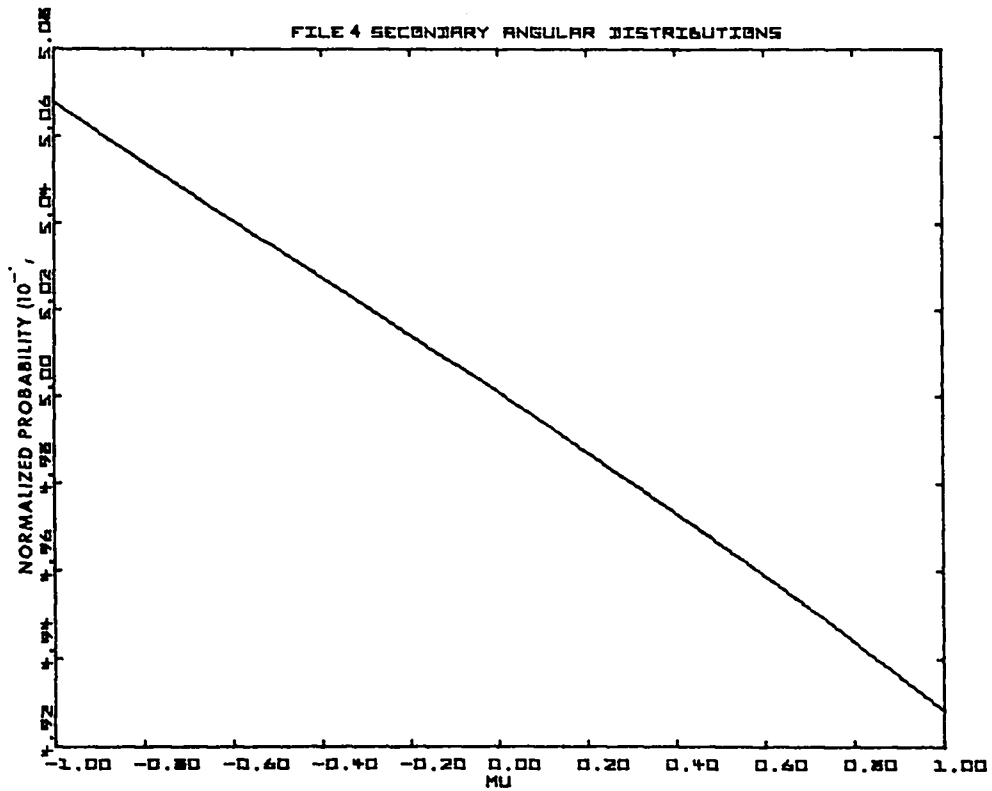


Fig. 8. Angular distribution for 4.0 MeV.

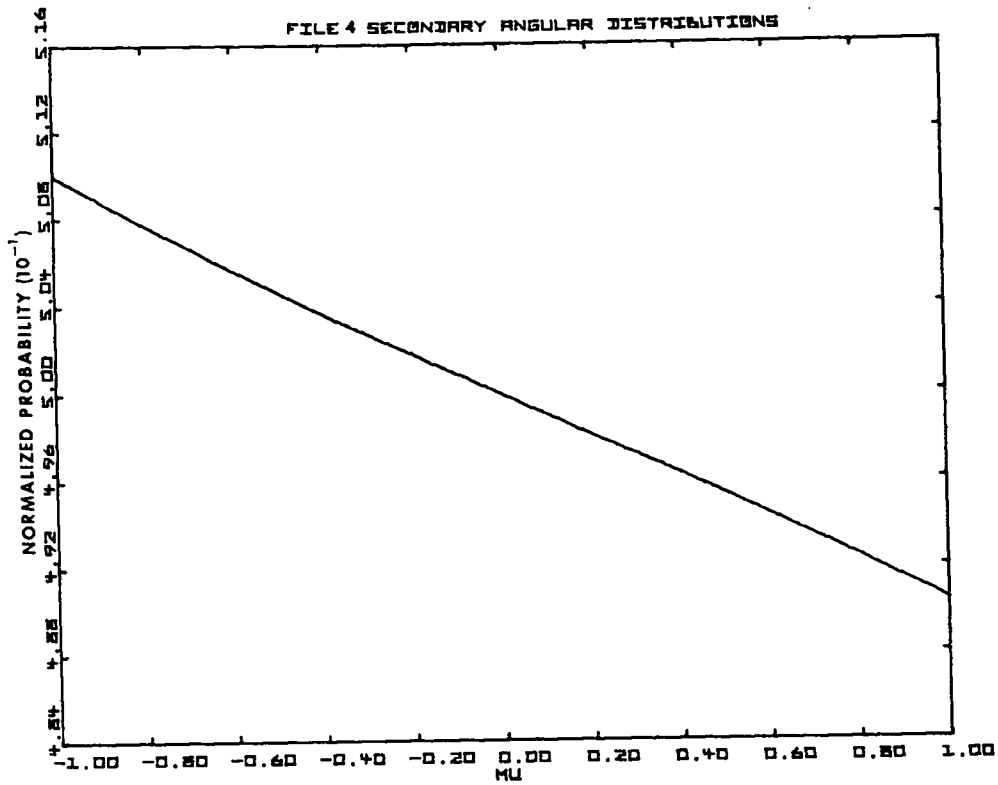


Fig. 9. Angular distribution for 6.0 MeV.

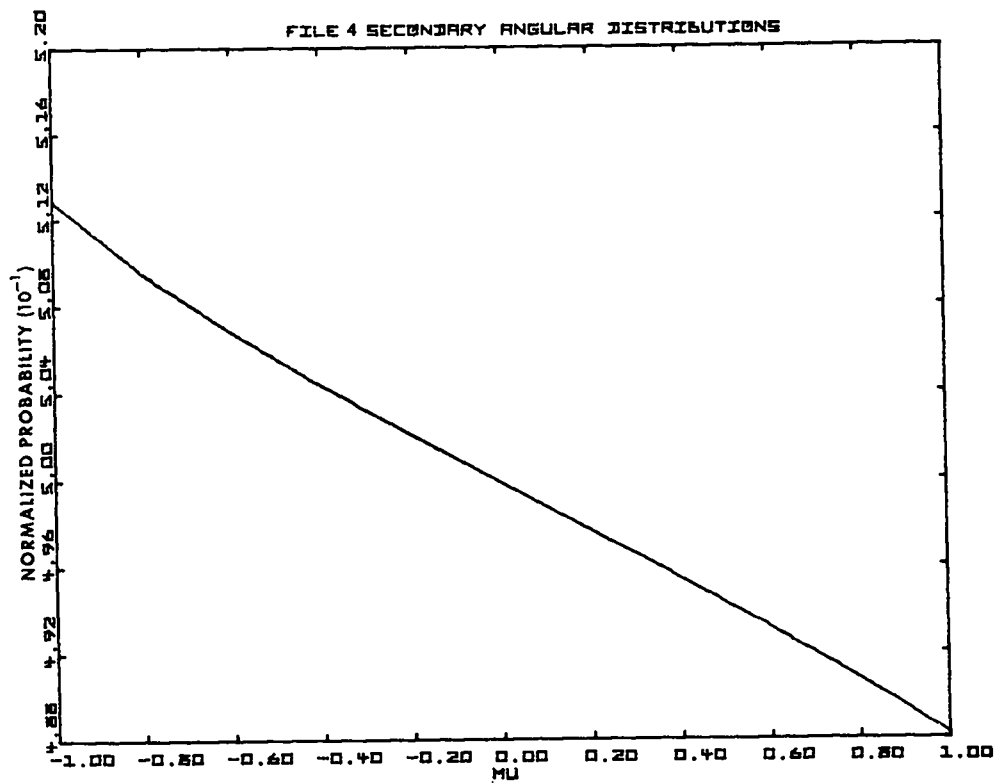


Fig. 10. Angular distribution for 8.0 MeV.

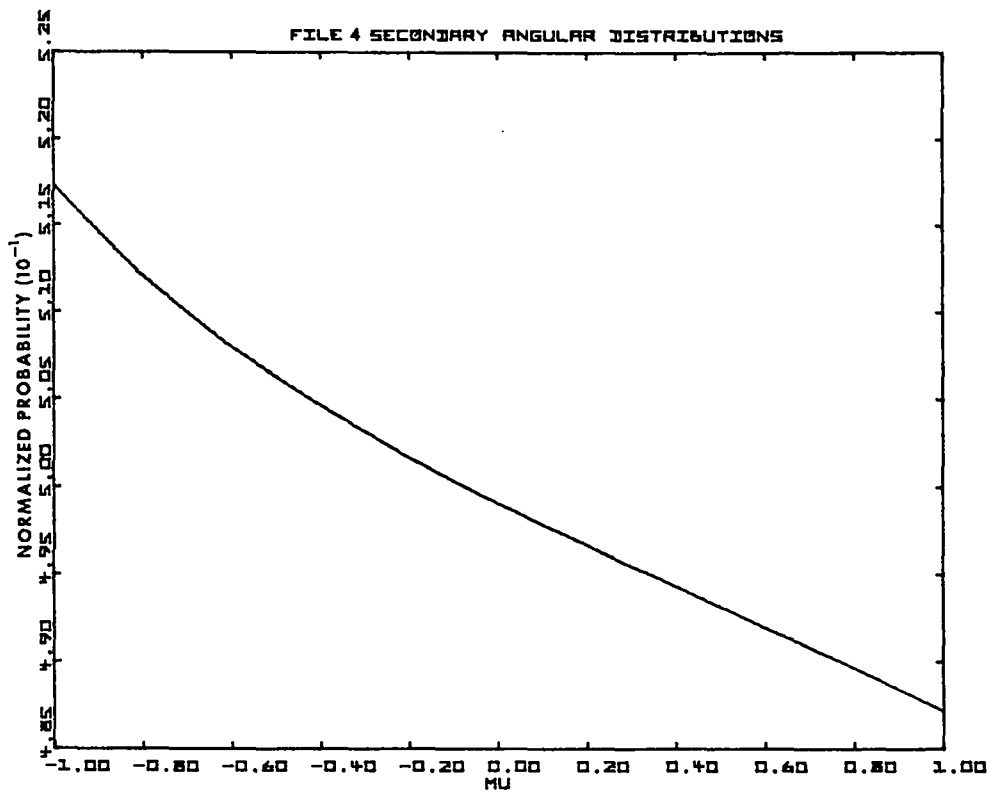


Fig. 11. Angular distribution for 10.0 MeV.

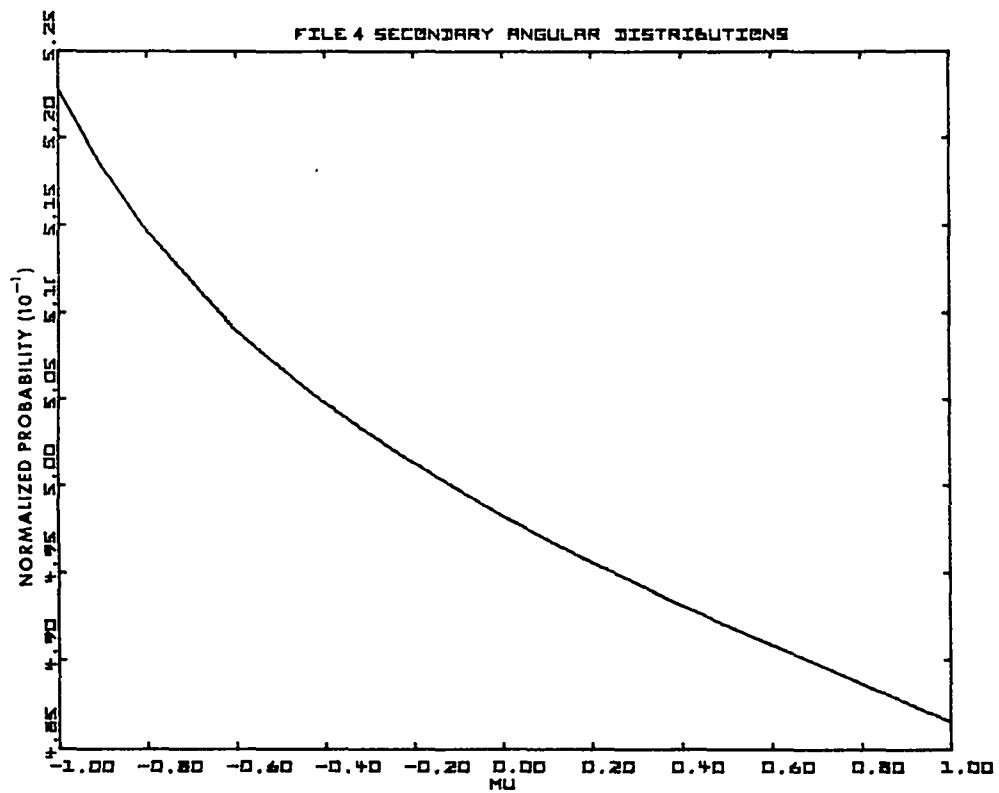


Fig. 12. Angular distribution for 12.0 MeV.

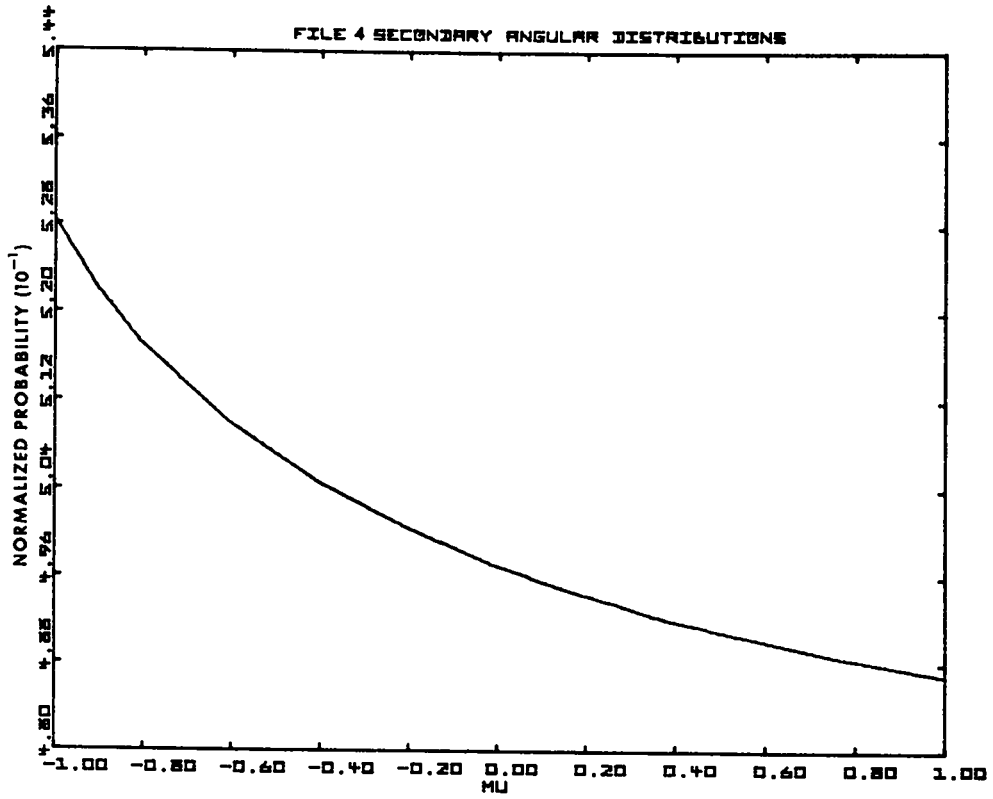


Fig. 13. Angular distribution for 14.0 MeV.

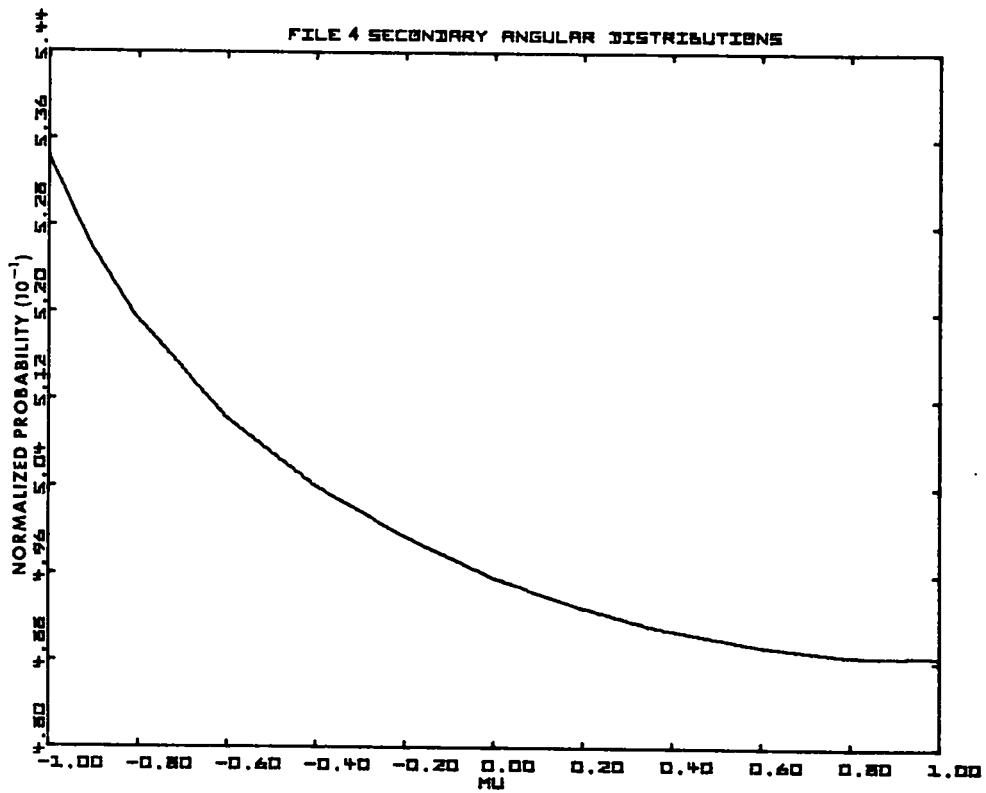


Fig. 14. Angular distribution for 16.0 MeV.

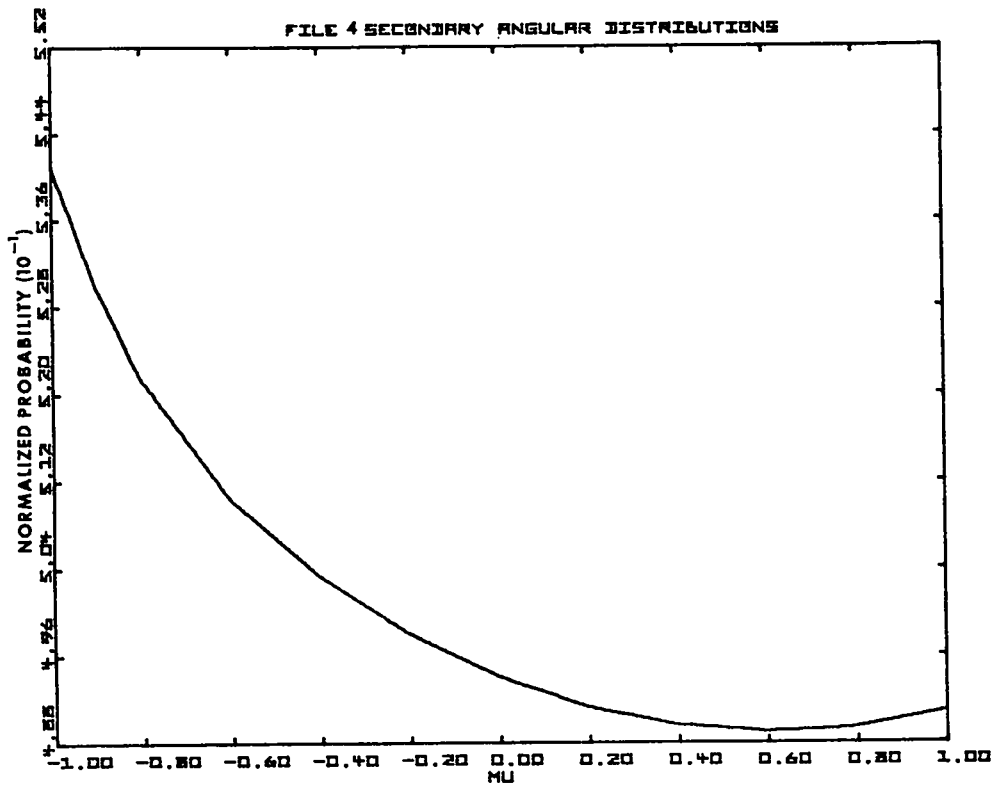


Fig. 15. Angular distribution for 18.0 MeV.

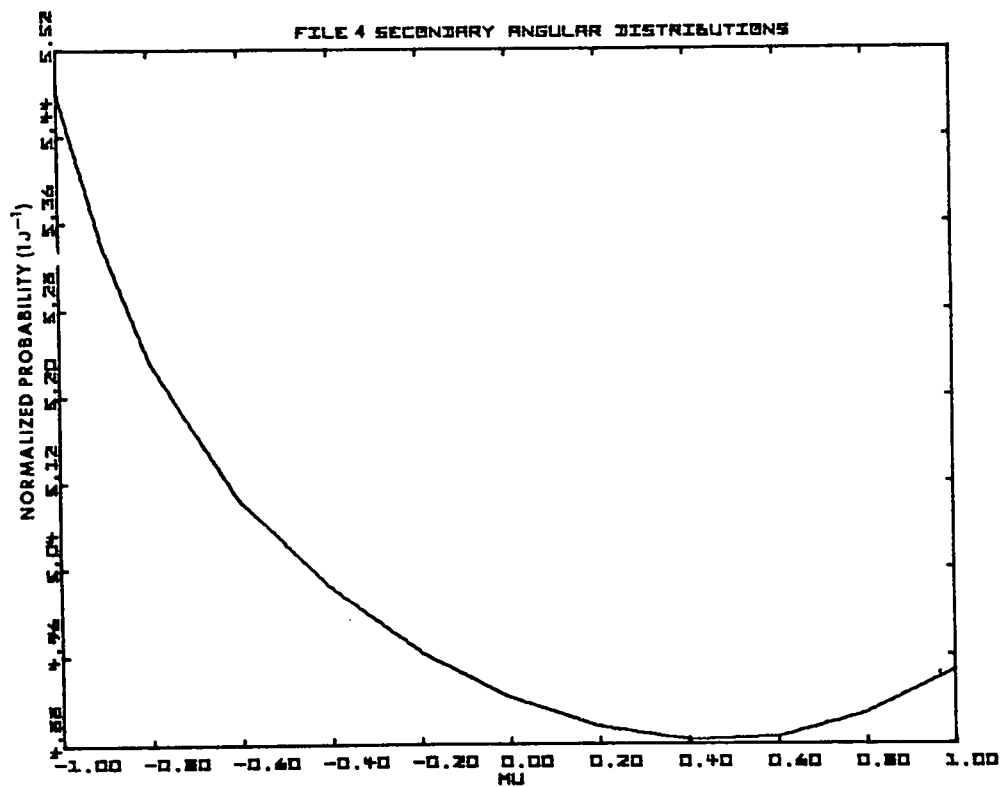


Fig. 16. Angular distribution for 20.0 MeV.

REFERENCES

1. J. C. Hopkins and G. Breit, "The H(n,n)H Scattering Observables Required for High Precision Fast-Neutron Measurements," submitted to Nuclear Data for publication; private communication prior to publication (1970).
2. A. Horsley, "Neutron Cross Sections of Hydrogen in the Energy Range 0.001 eV - 20 MeV," Nuclear Data A 2, 243 (1966) and private communication (1970).
3. "Chart of the Nuclides," Battelle Memorial Institute, May 1969.
4. R. E. Seamon, K. A. Friedman, G. Breit, R. D. Haracz, J. M. Holt, and A. Prakash, Phys. Rev. 165, 1579 (1968).
5. J. C. Davis and H. H. Barschall, "Adjustments in the n-p Singlet Effective Range," Phys. Letters 27B, 636 (1968).
6. J. C. Davis, BAPS 11 15, 474 (1970) and private communication (1970).

APPENDIX

LISTING OF HYDROGEN EVALUATION

HYDROGEN IN ENDF/B-II FORMAT. MAT = 1148						1148-0 -0	0
1.001 E+03	9.9917E-01	0	0	0		121148 1451	1
0.0	0.0	0	0	67		01148 1451	2
HYDROGEN FREE ATOM CROSS SECTIONS						1148 1451	3
ENTRY BY L. STEWART, R.J. LABAUVE, AND P.G. YOUNG						1148 1451	4
LOS ALAMOS SCIENTIFIC LABORATORY						1148 1451	5
LOS ALAMOS, NEW MEXICO 87544						1148 1451	6
OCTOBER 20, 1970						1148 1451	7
						1148 1451	8
						1148 1451	9
MF=1						1148 1451	10
MT=451. ATOMIC MASS=1.007825						1148 1451	11
						1148 1451	12
						1148 1451	13
MF=2						1148 1451	14
MT=151. SCATTERING LFNGTH=1.2756E-12 CM.						1148 1451	15
						1148 1451	16
						1148 1451	17
MF=3						1148 1451	18
						1148 1451	19
MT= 1. TOTAL CROSS SECTIONS --- THE TOTAL CROSS SECTIONS ARE						1148 1451	20
OBTAINED BY ADDING THE FLASTIC SCATTERING AND						1148 1451	21
RADIATIVE CAPTURE CROSS SECTIONS AT ALL ENERGIES.						1148 1451	22
1.0E-05 EV TO 20 MEV.						1148 1451	23
						1148 1451	24
MT= 2. ELASTIC SCATTERING --- FROM AN EXTENSIVE THEORETICAL						1148 1451	25
TREATMENT OF FAST NEUTRON MEASUREMENTS						1148 1451	26
BY J. C. HOPKINS(LASL) AND G. BREIT(STATE						1148 1451	27
UNIVERSITY OF NEW YORK).						1148 1451	28
1.0E-05 EV TO 20 MEV.						1148 1451	29
						1148 1451	30
MT=102. RADIATIVE CAPTURE --- THESE CROSS SECTIONS ARE TAKEN						1148 1451	31
FROM THE 1966 PUBLICATION OF A. HORSLEY WHERE A VALUE						1148 1451	32
OF 332 MB WAS ADOPTED FOR THE THERMAL VALUE.						1148 1451	33
1.0E-05 EV TO 20 MEV.						1148 1451	34
						1148 1451	35
MT=251. AVERAGE VALUF OF COSINE OF SCATTERING ANGLE						1148 1451	36
IN LAB SYSTEM.						1148 1451	37
1.0E-05 EV TO 20 MEV.						1148 1451	38
						1148 1451	39
MT=252. AVERAGE LOGARITHMIC ENERGY CHANGE PER COLLISON, TAKEN						1148 1451	40
AS 1, FROM 1.0E-05 EV TO 20 MEV.						1148 1451	41
						1148 1451	42
MT=253. GAMMA, TAKEN AS 1, FROM 1.0E-05 EV TO 20 MEV.						1148 1451	43
						1148 1451	44
MF=4						1148 1451	45
						1148 1451	46
MT= 2. NEUTRON ELASTIC SCATTERING ANGULAR DISTRIBUTIONS IN						1148 1451	47
THE CENTER OF MASS SYSTEM--GIVEN AS NORMALIZED						1148 1451	48
POINTWISE PROBABILITIES.						1148 1451	49

					1148 1451	50
					1148 1451	51
MF=7					1148 1451	52
					1148 1451	53
	MT= 4, .00001 TO 5 EV FREE GAS SIGMA=20.449 BARNS.				1148 1451	54
					1148 1451	55
MF=12					1148 1451	56
					1148 1451	57
	MT=102, GAMMA RAY MULTIPLICITIES --- MULTIPLICITY, (REFERRED TO MT=102, MF=3), IS UNITY AT ALL NEUTRON ENERGIES.				1148 1451	58
	SIXTEEN ENERGY BANDS ARE GIVEN FROM .2 MEV TO 20 MEV,				1148 1451	59
	AND THE AVERAGE GAMMA RAY ENERGY, EAG, IS DETERMINED				1148 1451	60
	FROM THE AVERAGE NEUTRON ENERGY, EAN, IN THE BAND BY				1148 1451	61
	EAG=2.225E+06+EAN/2., RECOIL ENERGY IGNORED.				1148 1451	62
					1148 1451	63
					1148 1451	64
MF=14					1148 1451	65
					1148 1451	66
	MT=102, GAMMA RAY ANGULAR DISTRIBUTION --- ASSUMED ISOTROPIC				1148 1451	67
	HYDROGEN IN ENDF/B-II FORMAT. MAT = 1148				1148-0 -0	0
1.001 E+03 9.9917E-01	0	0	0		121148 1451	1
0.0 0.0 0 0 67					01148 1451	2
					1148 1451	3
	HYDROGEN FREE ATOM CROSS SECTIONS				1148 1451	4
	ENTRY BY L. STEWART, R.J. LABAUVE, AND P.G. YOUNG				1148 1451	5
	LOS ALAMOS SCIENTIFIC LABORATORY				1148 1451	6
	LOS ALAMOS, NEW MEXICO 87544				1148 1451	7
	OCTOBER 20, 1970				1148 1451	8
					1148 1451	9
MF=1					1148 1451	10
					1148 1451	11
	MT=451, ATOMIC MASS=1.007825				1148 1451	12
					1148 1451	13
MF=2					1148 1451	14
					1148 1451	15
	MT=151, SCATTERING LENGTH=1.2756E-12 CM.				1148 1451	16
					1148 1451	17
MF=3					1148 1451	18
					1148 1451	19
	MT= 1, TOTAL CROSS SECTIONS --- THE TOTAL CROSS SECTIONS ARE OBTAINED BY ADDING THE ELASTIC SCATTERING AND RADIATIVE CAPTURE CROSS SECTIONS AT ALL ENERGIES.				1148 1451	20
	1.0E-05 EV TO 20 MEV.				1148 1451	21
					1148 1451	22
					1148 1451	23
					1148 1451	24
	MT= , ELASTIC SCATTERING --- FROM AN EXTENSIVE THEORETICAL TREATMENT OF FAST NEUTRON MEASUREMENTS BY J. C. HOPKINS(LASL) AND G. BREIT(STATE UNIVERSITY OF NEW YORK).				1148 1451	25
	1.0E-05 EV TO 20 MEV.				1148 1451	26
					1148 1451	27
					1148 1451	28
					1148 1451	29
					1148 1451	30
	MT=102, RADIATIVE CAPTURE --- THESE CROSS SECTIONS ARE TAKEN FROM THE 1966 PUBLICATION OF A. HORSLEY WHERE A VALUE OF 332 MB WAS ADOPTED FOR THE THERMAL VALUE.				1148 1451	31
	1.0E-05 EV TO 20 MEV.				1148 1451	32
					1148 1451	33
					1148 1451	34
					1148 1451	35
	MT=251, AVERAGE VALUF OF COSINE OF SCATTERING ANGLE IN LAB SYSTEM.				1148 1451	36
	1.0E-05 EV TO 20 MEV.				1148 1451	37
					1148 1451	38
					1148 1451	39
	MT=252, AVERAGE LOGARITHMIC ENERGY CHANGE PER COLLISON, TAKEN AS 1, FROM 1.0E-05 EV TO 20 MEV.				1148 1451	40
					1148 1451	41
					1148 1451	42
	MT=253, GAMMA, TAKEN AS 1, FROM 1.0E-05 EV TO 20 MEV.				1148 1451	43
					1148 1451	44
MF=4					1148 1451	45
					1148 1451	46
	MT= 2, NEUTRON ELASTIC SCATTERING ANGULAR DISTRIBUTIONS IN THE CENTER OF MASS SYSTEM--GIVEN AS NORMALIZED POINTWISE PROBABILITIES.				1148 1451	47
					1148 1451	48
					1148 1451	49

						1148	1451	50
						1148	1451	51
MF=7						1148	1451	52
	MT= 4. .00001 TO 5 FV FREE GAS SIGMA=20.449 BARNS.					1148	1451	53
						1148	1451	54
						1148	1451	55
MF=12						1148	1451	56
						1148	1451	57
	MT=102. GAMMA RAY MULTIPLICITIES --- MULTIPLICITY, (REFERRED					1148	1451	58
	TO MT=102. MF=3), IS UNITY AT ALL NEUTRON ENERGIES.					1148	1451	59
	SIXTEEN ENERGY BANDS ARE GIVEN FROM .2 MEV TO 20 MEV.					1148	1451	60
	AND THE AVERAGE GAMMA RAY ENERGY, EAG, IS DETERMINED					1148	1451	61
	FROM THE AVERAGE NEUTRON ENERGY, EAN, IN THE BAND BY					1148	1451	62
	EAG=2.225E+06*EAN/2., RECOIL ENERGY IGNORED.					1148	1451	63
						1148	1451	64
MF=14						1148	1451	65
						1148	1451	66
	MT=102. GAMMA RAY ANGULAR DISTRIBUTION --- ASSUMED ISOTROPIC					1148	1451	67
	AT ALL NEUTRON ENERGIES.					1148	1451	68
						1148	1451	69
		1	451	81		1148	1451	70
		2	151	4		1148	1451	71
		3	1	48		1148	1451	72
		3	2	48		1148	1451	73
		3	102	48		1148	1451	74
		3	251	8		1148	1451	75
		3	252	4		1148	1451	76
		3	253	4		1148	1451	77
		4	2	88		1148	1451	78
		7	4	4		1148	1451	79
		12	102	70		1148	1451	80
		14	102	1		1148	1451	81
						1148	1	0
						1148	0	0
						01148	2151	84
1.001 F+03 9.9917E-01		0	0	1		01148	2151	85
1.001 E+03 1.0000E+00		0	0	1		01148	2151	86
1.0 F-05 1.0 E+05		0	0	0		01148	2151	87
5.0 F-01 1.2756E+00		0	0	0		01148	2	0
0.0 0.0		0	0	0		01148	0	0
0.0 0.0		0	0	0		01148	3	1
1.001 F+03 9.9917E-01		0	0	0		1341148	3	1
0.0 0.0		0	0	1		1148	3	1
	134 5					1148	3	1
1.0000E-05 3.7148E+01 2.0000E-05 3.2257E+01 5.0000E-05 2.7917E+01						1148	3	1
1.0000E-04 2.5730E+01 2.0000E-04 2.4183E+01 5.0000E-04 2.2811E+01						1148	3	1
1.0000E-03 2.2119E+01 2.0000E-03 2.1630E+01 5.0000E-03 2.1196E+01						1148	3	1
1.0000E-02 2.0977E+01 2.5300E-02 2.0780E+01 1.0000E+02 2.0450E+01						1148	3	1
1.0000E+03 2.0330E+01 2.0000E+03 2.0200E+01 3.0000E+03 2.0070E+01						1148	3	1
4.0000E+03 1.9940E+01 5.0000E+03 1.9820E+01 6.0000E+03 1.9690E+01						1148	3	1
8.0000E+03 1.9450E+01 1.0000E+04 1.9210E+01 1.5000E+04 1.8650E+01						1148	3	1
2.0000E+04 1.8130E+01 2.5000E+04 1.7630E+01 3.0000E+04 1.7170E+01						1148	3	1
3.5000E+04 1.6740E+01 4.0000E+04 1.6330E+01 4.5000E+04 1.5940E+01						1148	3	1
5.0000E+04 1.5580E+01 5.5000E+04 1.5230E+01 6.0000E+04 1.4900E+01						1148	3	1
6.5000E+04 1.4590E+01 7.0000E+04 1.4290E+01 7.5000E+04 1.4010E+01						1148	3	1
8.0000E+04 1.3740E+01 8.5000E+04 1.3480E+01 9.0000E+04 1.3240E+01						1148	3	1
9.5000E+04 1.3000E+01 1.0000E+05 1.2770E+01 1.1000E+05 1.2350E+01						1148	3	1
1.2000E+05 1.1960E+01 1.3000E+05 1.1610E+01 1.4000E+05 1.1280E+01						1148	3	1
1.5000E+05 1.0970E+01 1.6000E+05 1.0670E+01 1.7000E+05 1.0400E+01						1148	3	1
1.8000E+05 1.0140E+01 1.9000E+05 9.8980E+00 2.0000E+05 9.6710E+00						1148	3	1
2.2000E+05 9.2580E+00 2.4000E+05 8.8920E+00 2.6000E+05 8.5620E+00						1148	3	1
2.8000E+05 8.2620E+00 3.0000E+05 7.9870E+00 3.2000E+05 7.7340E+00						1148	3	1
3.4000E+05 7.5010E+00 3.6000E+05 7.2840E+00 3.8000E+05 7.0830E+00						1148	3	1
4.0000E+05 6.8970E+00 4.2000E+05 6.7250E+00 4.4000E+05 6.5650E+00						1148	3	1
4.6000E+05 6.4150E+00 4.8000E+05 6.2750E+00 5.0000E+05 6.1430E+00						1148	3	1
5.5000E+05 5.8450E+00 6.0000E+05 5.5840E+00 6.5000E+05 5.3540E+00						1148	3	1
7.0000E+05 5.1480E+00 7.5000E+05 4.9640E+00 8.0000E+05 4.7970E+00						1148	3	1
8.5000E+05 4.6450E+00 9.0000E+05 4.5060E+00 9.5000E+05 4.3780E+00						1148	3	1
1.0000E+06 4.2610E+00 1.1000E+06 4.0510E+00 1.2000E+06 3.8680E+00						1148	3	1
1.3000E+06 3.7060E+00 1.4000E+06 3.5610E+00 1.5000E+06 3.4290E+00						1148	3	1
1.6000E+06 3.3090E+00 1.7000E+06 3.1980E+00 1.8000E+06 3.0970E+00						1148	3	1





1.0000E-05	1.6699E+01	2.0000E-05	1.1808E+01	5.0000E-05	7.4682E+00	1148	3102	191
1.0000E-04	5.2808E+00	2.0000E-04	3.7341E+00	5.0000E-04	2.3616E+00	1148	3102	192
1.0000E-03	1.6699E+00	2.0000E-03	1.1808E+00	5.0000E-03	7.4682E-01	1148	3102	193
1.0000E-02	5.2808E-01	2.5300E-02	3.3200E-01	1.0000E+02	5.2770E-03	1148	3102	194
1.0000E+03	1.6590E-03	2.0000E+03	1.1926E-03	3.0000E+03	9.7008E-04	1148	3102	195
4.0000E+03	8.3290E-04	5.0000E+03	7.3747E-04	6.0000E+03	6.6619E-04	1148	3102	196
8.0000E+03	5.6518E-04	1.0000E+04	4.9580E-04	1.5000E+04	3.8782E-04	1148	3102	197
2.0000E+04	3.2386E-04	1.5000E+04	6.8064E-04	3.0000E+04	2.4909E-04	1148	3102	198
3.5000E+04	2.2248E-04	4.0000E+04	2.0553E-04	4.5000E+04	1.8970E-04	1148	3102	199
5.0000E+04	1.7646E-04	5.5000E+04	1.6518E-04	6.0000E+04	1.5544E-04	1148	3102	200
6.5000E+04	1.4693E-04	7.0000E+04	1.3942E-04	7.5000E+04	1.3273E-04	1148	3102	201
8.0000E+04	1.2673E-04	8.5000E+04	1.2132E-04	9.0000E+04	1.1640E-04	1148	3102	202
9.5000E+04	1.1191E-04	1.0000E+05	1.0780E-04	1.1000E+05	1.0019E-04	1148	3102	203
1.2000E+05	9.3717E-05	1.3000E+05	8.8131E-05	1.4000E+05	8.3256E-05	1148	3102	204
1.5000E+05	7.8960E-05	1.6000E+05	7.5142E-05	1.7000E+05	7.1725E-05	1148	3102	205
1.8000E+05	6.8645E-05	1.9000E+05	6.5853E-05	2.0000E+05	6.3310E-05	1148	3102	206
2.2000E+05	5.9051E-05	2.4000E+05	5.5591E-05	2.6000E+05	5.2731E-05	1148	3102	207
2.8000E+05	5.0330E-05	3.0000E+05	4.8290E-05	3.2000E+05	4.6538E-05	1148	3102	208
3.4000E+05	4.5018E-05	3.6000E+05	4.3691E-05	3.8000E+05	4.2523E-05	1148	3102	209
4.0000E+05	4.1490E-05	4.2000E+05	4.0607E-05	4.4000E+05	3.9828E-05	1148	3102	210
4.6000E+05	3.9138E-05	4.8000E+05	3.8525E-05	5.0000E+05	3.7980E-05	1148	3102	211
5.5000E+05	3.7396E-05	6.0000E+05	3.6870E-05	6.5000E+05	3.6163E-05	1148	3102	212
7.0000E+05	3.5520E-05	7.5000E+05	3.5167E-05	8.0000E+05	3.4840E-05	1148	3102	213
8.5000E+05	3.4742E-05	9.0000E+05	3.4650E-05	9.5000E+05	3.4552E-05	1148	3102	214
1.0000E+06	3.4460E-05	1.1000E+06	3.4440E-05	1.2000E+06	3.4410E-05	1148	3102	215
1.3000E+06	3.4490E-05	1.4000E+06	3.4360E-05	1.5000E+06	3.4340E-05	1148	3102	216
1.6000E+06	3.4310E-05	1.7000E+06	3.4290E-05	1.8000E+06	3.4270E-05	1148	3102	217
1.9000E+06	3.4250E-05	2.0000E+06	3.4230E-05	2.2000E+06	3.4520E-05	1148	3102	218
2.4000E+06	3.4810E-05	2.6000E+06	3.5100E-05	2.8000E+06	3.5390E-05	1148	3102	219
3.0000E+06	3.5680E-05	3.2000E+06	3.5800E-05	3.4000E+06	3.5910E-05	1148	3102	220
3.6000E+06	3.6030E-05	3.8000E+06	3.6140E-05	4.0000E+06	3.6260E-05	1148	3102	221
4.2000E+06	3.6290E-05	4.4000E+06	3.6320E-05	4.6000E+06	3.6360E-05	1148	3102	222
4.8000E+06	3.6390E-05	5.0000E+06	3.6420E-05	5.2000E+06	3.6290E-05	1148	3102	223
5.4000E+06	3.6160E-05	5.6000E+06	3.6040E-05	5.8000E+06	3.5910E-05	1148	3102	224
6.0000E+06	3.5780E-05	6.2000E+06	3.5670E-05	6.4000E+06	3.5560E-05	1148	3102	225
6.6000E+06	3.5450E-05	6.8000E+06	3.5340E-05	7.0000E+06	3.5230E-05	1148	3102	226
7.5000E+06	3.4590E-05	8.0000E+06	3.3940E-05	8.5000E+06	3.3640E-05	1148	3102	227
9.0000E+06	3.3330E-05	9.5000E+06	3.2960E-05	1.0000E+07	3.2590E-05	1148	3102	228
1.0500E+07	3.2210E-05	1.1000E+07	3.1820E-05	1.1500E+07	3.1450E-05	1148	3102	229
1.2000E+07	3.1080E-05	1.2500E+07	3.0630E-05	1.3000E+07	3.0180E-05	1148	3102	230
1.3500E+07	3.0010E-05	1.4000E+07	2.9830E-05	1.4500E+07	2.9400E-05	1148	3102	231
1.5000E+07	2.8960E-05	1.5500E+07	2.8630E-05	1.6000E+07	2.8300E-05	1148	3102	232
1.6500E+07	2.7880E-05	1.7000E+07	2.7450E-05	1.7500E+07	2.7360E-05	1148	3102	233
1.8000E+07	2.7260E-05	1.8500E+07	2.6730E-05	1.9000E+07	2.6200E-05	1148	3102	234
1.9500E+07	2.6120E-05	2.0000E+07	2.6040E-05	-0.	-0.	1148	3102	235
						1148	3	0
1.001 E+03	9.9917E-01		0	0	0	01148	3251	237
0.0	0.0		0	0	1	141148	3251	238
	14	3				1148	3251	239
1.0	E-056.65213E-01	1.0	E+056.65213E-01	5.0	E+056.64899E-01	1148	3251	240
1.0	F+066.64620E-01	2.0	E+066.64149F-01	4.0	E+066.63355E-01	1148	3251	241
6.0	E+066.62628E-01	8.0	E+066.62018E-01	1.0	E+076.61338E-01	1148	3251	242
1.2	F+076.61045E-01	1.4	E+076.60449E-01	1.6	F+076.59929E-01	1148	3251	243
1.8	E+076.59540E-01	2.0	E+076.59196E-01			1148	3251	244
						1148	3	0
1.001 F+03	9.9917E-01		0	0	0	01148	3252	246
0.0	0.0		0	0	1	21148	3252	247
	2	2				1148	3252	248
1.0	E-05 1.0	2.0	E+07 1.0			1148	3252	249
						1148	3	0
1.001 E+03	9.9917E-01		0	0	0	01148	3253	251
0.0	0.0		0	0	1	21148	3253	252
	2	2				1148	3253	253
1.0	E-05 1.0	2.0	E+07 1.0			1148	3253	254
						1148	3	0
						1148	0	0
1.001 E+03	9.9917E-01		0	2	0	01148	4	2
0.0	0.0		0	2	0	01148	4	2
	0.0	0.0	0	0	1	141148	4	2
	14	2				1148	4	2

0.0	1.0-05	0	0	1	111148	4	2	261	
11	2				1148	4	2	262	
-1.0	.5	-.8	.5	-.6	.51148	4	2	263	
-.4	.5	-.2	.5	0.	.51148	4	2	264	
.2	.5	.4	.5	.6	.51148	4	2	265	
.8	.5	1.0	.5		1148	4	2	266	
0.0	1.0+05	0	0	1	111148	4	2	267	
11	2				1148	4	2	268	
-1.0	.5	-.8	.5	-.6	.51148	4	2	269	
-.4	.5	-.2	.5	0.	.51148	4	2	270	
.2	.5	.4	.5	.6	.51148	4	2	271	
.8	.5	1.0	.5		1148	4	2	272	
0.0	5.0+05	0	0	1	111148	4	2	273	
11	2				1148	4	2	274	
-.10000E+01	.50117E+00	-.80000E+00	.50097E+00	-.60000E+00	.50066E+00	001148	4	2	275
-.40000E+00	.50045E+00	-.20000E+00	.50025E+00		.50005E+00	001148	4	2	276
.20000E+00	.49974E+00	.40000E+00	.49953E+00	.60000E+00	.49933E+00	001148	4	2	277
.80000E+00	.49902E+00	.10000E+01	.49882E+00			1148	4	2	278
0.0	1.0+06	0	0	1	111148	4	2	279	
11	2				1148	4	2	280	
-.10000E+01	.50218E+00	-.80000E+00	.50176E+00	-.60000E+00	.50131E+00	001148	4	2	281
-.40000E+00	.50089E+00	-.20000E+00	.50044E+00		.50000E+00	001148	4	2	282
.20000E+00	.49957E+00	.40000E+00	.49913E+00	.60000E+00	.49869E+00	001148	4	2	283
.80000E+00	.49823E+00	.10000E+01	.49779E+00			1148	4	2	284
0.0	2.0+06	0	0	1	111148	4	2	285	
11	2				1148	4	2	286	
-.10000E+01	.50387E+00	-.80000E+00	.50311E+00	-.60000E+00	.50234E+00	001148	4	2	287
-.40000E+00	.50158E+00	-.20000E+00	.50081E+00		.50003E+00	001148	4	2	288
.20000E+00	.49925E+00	.40000E+00	.49846E+00	.60000E+00	.49764E+00	001148	4	2	289
.80000E+00	.49682E+00	.10000E+01	.49598E+00			1148	4	2	290
0.0	4.0+06	0	0	1	111148	4	2	291	
11	2				1148	4	2	292	
-.10000E+01	.50677E+00	-.80000E+00	.50542E+00	-.60000E+00	.50407E+00	001148	4	2	293
-.40000E+00	.50275E+00	-.20000E+00	.50143E+00		.50011E+00	001148	4	2	294
.20000E+00	.49873E+00	.40000E+00	.49734E+00	.60000E+00	.49589E+00	001148	4	2	295
.80000E+00	.49441E+00	.10000E+01	.49283E+00			1148	4	2	296
0.0	6.0+06	0	0	1	111148	4	2	297	
11	2				1148	4	2	298	
-.10000E+01	.50999E+00	-.80000E+00	.50770E+00	-.60000E+00	.50560E+00	001148	4	2	299
-.40000E+00	.50362E+00	-.20000E+00	.50177E+00		.49993E+00	001148	4	2	300
.20000E+00	.49808E+00	.40000E+00	.49628E+00	.60000E+00	.49439E+00	001148	4	2	301
.80000E+00	.49246E+00	.10000E+01	.49040E+00			1148	4	2	302
0.0	8.0+06	0	0	1	111148	4	2	303	
11	2				1148	4	2	304	
-.10000E+01	.51288E+00	-.80000E+00	.50952E+00	-.60000E+00	.50676E+00	001148	4	2	305
-.40000E+00	.50434E+00	-.20000E+00	.50207E+00		.49987E+00	001148	4	2	306
.20000E+00	.49766E+00	.40000E+00	.49546E+00	.60000E+00	.49314E+00	001148	4	2	307
.80000E+00	.49077E+00	.10000E+01	.48818E+00			1148	4	2	308
0.0	10.0+06	0	0	1	111148	4	2	309	
11	2				1148	4	2	310	
-.10000E+01	.51727E+00	-.80000E+00	.51201E+00	-.60000E+00	.50794E+00	001148	4	2	311
-.40000E+00	.50461E+00	-.20000E+00	.50168E+00		.49908E+00	001148	4	2	312
.20000E+00	.49669E+00	.40000E+00	.49435E+00	.60000E+00	.49202E+00	001148	4	2	313
.80000E+00	.48969E+00	.10000E+01	.48716E+00			1148	4	2	314
0.0	12.0+06	0	0	1	121148	4	2	315	
12	2				1148	4	2	316	
-.10000E+01	.52272E+00	-.90000E+00	.51825E+00	-.80000E+00	.51464E+00	001148	4	2	317
-.60000E+00	.50898E+00	-.40000E+00	.50475E+00	-.20000E+00	.50129E+00	001148	4	2	318
0.	.49823E+00	.20000E+00	.49556E+00	.40000E+00	.49313E+00	001148	4	2	319
.60000E+00	.49085E+00	.80000E+00	.48866E+00	.10000E+01	.48654E+00	001148	4	2	320
0.0	14.0+06	0	0	1	121148	4	2	321	
12	2				1148	4	2	322	
-.10000E+01	.52823E+00	-.90000E+00	.52188E+00	-.80000E+00	.51698E+00	001148	4	2	323
-.60000E+00	.50982E+00	-.40000E+00	.50456E+00	-.20000E+00	.50048E+00	001148	4	2	324
0.	.49703E+00	.20000E+00	.49431E+00	.40000E+00	.49195E+00	001148	4	2	325
.60000E+00	.49005E+00	.80000E+00	.48833E+00	.10000E+01	.48688E+00	001148	4	2	326
0.0	16.0+06	0	0	1	121148	4	2	327	
12	2				1148	4	2	328	
-.10000E+01	.53433E+00	-.90000E+00	.52575E+00	-.80000E+00	.51924E+00	001148	4	2	329
-.60000E+00	.51024E+00	-.40000E+00	.50404E+00	-.20000E+00	.49939E+00	001148	4	2	330

0.	.49567E+00	.20000E+00	.49288E+00	.40000E+00	.49081E+00	1148	4	2	331
.60000E+00	.48936E+00	.80000E+00	.48853E+00	.10000E+01	.48833E+00	1148	4	2	332
0.0	18.0+06	0	0	0	1	121148	4	2	333
12	2					1148	4	2	334
-.10000E+01	.54092E+00	-.40000E+00	.52962E+00	-.80000E+00	.52134E+00	1148	4	2	335
-.60000E+00	.51038E+00	-.40000E+00	.50327E+00	-.20000E+00	.49802E+00	1148	4	2	336
0.	.49406E+00	.20000E+00	.49126E+00	.40000E+00	.48963E+00	1148	4	2	337
.60000E+00	.48905E+00	.80000E+00	.48940E+00	.10000E+01	.49091E+00	1148	4	2	338
0.0	20.0+06	0	0	0	1	121148	4	2	339
12	2					1148	4	2	340
-.10000E+01	.54807E+00	-.90000E+00	.53348E+00	-.80000E+00	.52332E+00	1148	4	2	341
-.60000E+00	.51029E+00	-.40000E+00	.50221E+00	-.20000E+00	.49635E+00	1148	4	2	342
0.	.49218E+00	.20000E+00	.48958E+00	.40000E+00	.48840E+00	1148	4	2	343
.60000E+00	.48866E+00	.80000E+00	.49075E+00	.10000E+01	.49466E+00	1148	4	2	344
						1148	4	0	345
						1148	0	0	346
1.001 E+03	9.9917E-01	0	0	0	0	01148	7	4	347
0.0	0.0	0	0	0	12	11148	7	4	348
0.0	2.0 E+02	9.9917E-01	5.0	0.0	0.0	1148	7	4	349
1.0	2.0449E+01	9.9917E-01	0.0	0.0	0.0	1148	7	4	350
						1148	7	0	351
						1148	0	0	352
1.0010E+03	9.9917E-01	1	0	17		0114812102			353
0.	0.	0	0	1		2114812102			354
2	2					114812102			355
1.0000E-05	1.0000E+00	2.0000E+07	1.0000E+00			114812102			356
1.1725E+07	0.	0	2	1		3114812102			357
3	2					114812102			358
1.8000E+07	0.	1.8001E+07	1.0000E+00	2.0000E+07	1.0000E+00	114812102			359
1.0725E+07	0.	0	2	1		4114812102			360
4	2					114812102			361
1.6000E+07	0.	1.6001E+07	1.0000E+00	1.8000E+07	1.0000E+00	114812102			362
1.8001E+07	0.					114812102			363
9.7250E+06	0.	0	2	1		4114812102			364
4	2					114812102			365
1.4000E+07	0.	1.4001E+07	1.0000E+00	1.6000E+07	1.0000E+00	114812102			366
1.6001E+07	0.					114812102			367
8.7250E+06	0.	0	2	1		4114812102			368
4	2					114812102			369
1.2000E+07	0.	1.2001E+07	1.0000E+00	1.4000E+07	1.0000E+00	114812102			370
1.4001E+07	0.					114812102			371
7.7250E+06	0.	0	2	1		4114812102			372
4	2					114812102			373
1.0000E+07	0.	1.0001E+07	1.0000E+00	1.2000E+07	1.0000E+00	114812102			374
1.2001E+07	0.					114812102			375
6.9750E+06	0.	0	2	1		4114812102			376
4	2					114812102			377
9.0000E+06	0.	9.0001E+06	1.0000E+00	1.0000E+07	1.0000E+00	114812102			378
1.0001E+07	0.					114812102			379
6.4750E+06	0.	0	2	1		4114812102			380
4	2					114812102			381
8.0000E+06	0.	8.0001E+06	1.0000E+00	9.0000E+06	1.0000E+00	114812102			382
9.0001E+06	0.					114812102			383
5.9750E+06	0.	0	2	1		4114812102			384
4	2					114812102			385
7.0000E+06	0.	7.0001E+06	1.0000E+00	8.0000E+06	1.0000E+00	114812102			386
8.0001E+06	0.					114812102			387
5.4750E+06	0.	0	2	1		4114812102			388
4	2					114812102			389
6.0000E+06	0.	6.0001E+06	1.0000E+00	7.0000E+06	1.0000E+00	114812102			390
7.0001E+06	0.					114812102			391
4.9750E+06	0.	0	2	1		4114812102			392
4	2					114812102			393
5.0000E+06	0.	5.0001E+06	1.0000E+00	6.0000E+06	1.0000E+00	114812102			394
6.0001E+06	0.					114812102			395
4.4750E+06	0.	0	2	1		4114812102			396
4	2					114812102			397
4.0000E+06	0.	4.0001E+06	1.0000E+00	5.0000E+06	1.0000E+00	114812102			398
5.0001E+06	0.					114812102			399
3.9750E+06	0.	0	2	1		4114812102			400

	4	2				114812102	401
3.0000E+06	0.		3.0001E+06	1.0000E+00	4.0000E+06	1.0000E+00	114812102 402
4.0001E+06	0.						114812102 403
3.4750E+06	0.		0	2	1	4114812102	404
	4	2				114812102	405
2.0000E+06	0.		2.0001E+06	1.0000E+00	3.0000E+06	1.0000E+00	114812102 406
3.0001E+06	0.						114812102 407
2.9750E+06	0.		0	2	1	4114812102	408
	4	2				114812102	409
1.0000E+06	0.		1.0001E+06	1.0000E+00	2.0000E+06	1.0000E+00	114812102 410
2.0001E+06	0.						114812102 411
2.6250E+06	0.		0	2	1	4114812102	412
	4	2				114812102	413
6.0000E+05	0.		6.0001E+05	1.0000E+00	1.0000E+06	1.0000E+00	114812102 414
1.0001E+06	0.						114812102 415
2.4250E+06	0.		0	2	1	4114812102	416
	4	2				114812102	417
2.0000E+06	0.		2.0001E+05	1.0000E+00	6.0000E+05	1.0000E+00	114812102 418
6.0001E+05	0.						114812102 419
2.2250E+06	0.		0	2	1	3114812102	420
	3	2				114812102	421
1.0000E-05	1.0000E+00	2.0000E+05	1.0000E+00	2.0001E+05	0.	114812102	422
						114812	0 423
						1148	0 0 424
1.0010E+03	9.9917E-01	1	0	0		0114814102	425
						114814	0 426
						1148	0 0 427
						-1	428